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# HTGR Program Status

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**U.S. Department of Energy**

IAEA Technical Meeting

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# FY 2015 Budget Request for Reactor Concepts Research, Development, and Demonstration

## Budget Summary

\$ in thousands

Program Element	FY 2014 Enacted	FY 2015 Request
<b>Small Modular Reactor Advanced Concepts R&amp;D*</b>	<b>22,964</b>	--
<b>Light Water Reactor Sustainability</b>	<b>29,953</b>	<b>30,300</b>
<b>Advanced Reactor Technology</b>	<b>59,905</b>	<b>70,240</b>
<b>Total:</b>	<b>112,822</b>	<b>100,540</b>

\*SMR Advanced Concepts R&D included in Advanced Reactor Technology starting in FY 2015

## Mission

Develop new and advanced reactor designs and technologies that advance the state of reactor technology to improve competitiveness and support meeting Nation’s energy , environmental, and national security goals.

## FY 2015 Planned Accomplishments

- Release “Beta” version of the RELAP-7 code for industry review and testing to support the Risk Informed Safety Margin Characterization approach.
- Complete mechanistic models that predict Reactor Pressure Vessel embrittlement and irradiation-assisted stress corrosion cracking.
- Provide technical reports to NRC on General Design Criteria related topics and advanced reactor technologies to support the NRC’s establishment of an advanced reactor licensing framework.
- Perform TRISO post-irradiation examinations.
- Perform graphite irradiation and post - irradiation examination experiments.
- Complete ASME Code Qualification case proposals for selected advanced reactor materials.

# US HTGR Program Overview

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## ■ Ongoing priority research:

- AGR fuel qualification program
- AGC graphite qualification program
- High temperature materials qualification
- Passive heat removal system testing and modeling

## ■ Interactions with NRC

- Review and feedback on priority research activities
- Establishment of a modular HTGR licensing framework
  - Development of safety design criteria
  - Resolution of other key technical and policy issues ( requirements for functional containment, establishment of mechanistic source term, etc.)

## ■ Engaging with Industry

- Developing business models and deployment strategies



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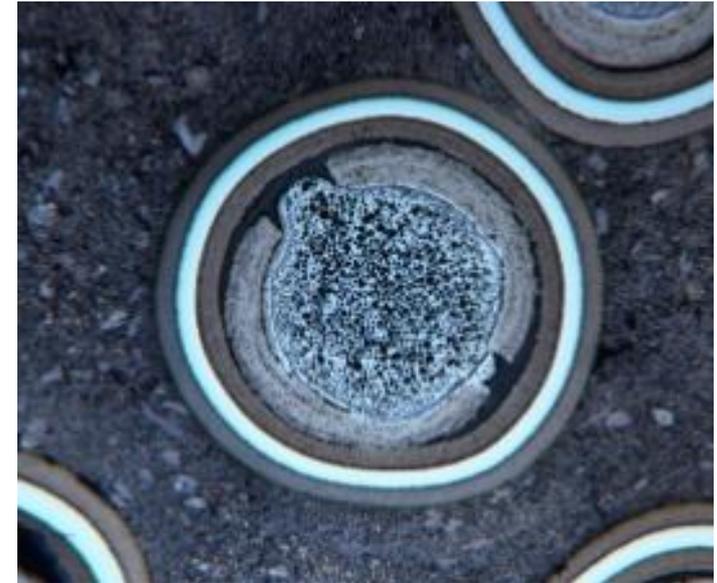
# Advanced Gas Reactor (AGR) Fuel Qualification Program

## ■ Ongoing Multi-Year AGR Program is Focused on Demonstrating:

- Fabrication of high-quality low-defect fuel is achievable at industrial scale
- Robust irradiation performance
- Robust accident performance

## ■ Draft NRC Feedback on AGR Program

- The scope of AGR Program activities is generally reasonable in context of pre-prototype testing
- Early AGR irradiation and safety testing results show promise for demonstrating much of desired tristructural isotropic (TRISO) fuel retention capability
- Additional data are needed from fuel and core testing in a prototype reactor to provide reasonable assurance of targeted fission product retention in fuel



**TRISO fuel forms heart of  
HTGR safety case**



## Materials Testing and Qualification

### ■ High Temperature Materials Program

- Alloy 800H is being qualified for extended operation up to 850°C and Alloy 617 up to 950°C, which will be adequate for the modular HTGR.
- Improvements in high temperature design methodology are underway to reassess and improve the design margins needed for HTGRs.

### ■ AGC – Graphite Testing Program

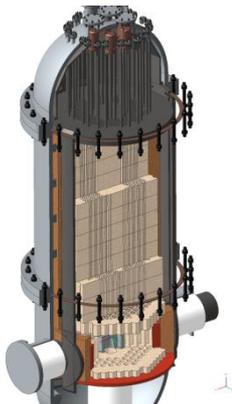
- Develop the qualification data set of thermo-mechanical and thermo-physical properties for un-irradiated and irradiated candidate grades of graphite for HTGRs.
- Demonstrate the performance of various graphite types under bounding conditions, including irradiation dose levels, anticipated applied stress levels, and maximum core temperatures.
- Provide an improved understanding of graphite irradiation and environmental degradation mechanisms for use in developing improved models of graphite behavior for design and performance.

- Continuing to support computational fluid dynamics work as part of the IAEA Coordinated Research Program (CRP) on High Temperature Gas-cooled Reactor Uncertainties, utilizing the commercial code CFX
  - Results to be summarized in a High Temperature Reactor 2014 conference paper on the overall progress status of Phase I
- Established second workshop of the OECD Nuclear Energy Agency's Benchmark for Prismatic Coupled Neutronics/Thermal Fluids Transient of the Modular High-Temperature Gas-Cooled Reactor 350 MW Core Design
  - June 23-26, 2014 at Idaho National Laboratory
- Developing experimental datasets that envelop the anticipated prismatic HTGR operational regime
- Verifying and validating assessment codes and simulation models for generic HTGR designs and computational benchmarks

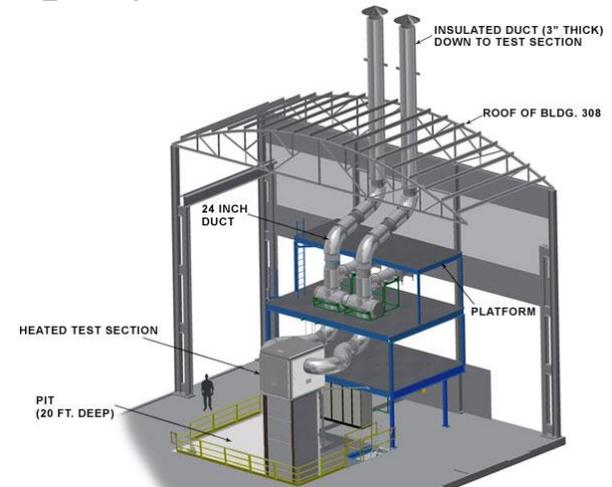


# Passive Heat Removal System – Testing Activities

- The capability to remove reactor core residual heat by passive and inherent means is another key to the safety basis for the modular HTGR design concept.
- The NRC’s Policy Statement on Advanced Reactors provides an expectation that advanced reactors will provide enhanced margins of safety, including use of:  
***“Highly reliable and less complex shutdown and decay heat removal systems. The use of inherent or passive means to accomplish this objective is encouraged.”***
- The US Department of Energy is supporting the testing of passive heat removal systems and concepts:



- » Argonne National Laboratory
- » Oregon State University
- » Texas A&M University
- » University of Idaho
- » University of Wisconsin – Madison





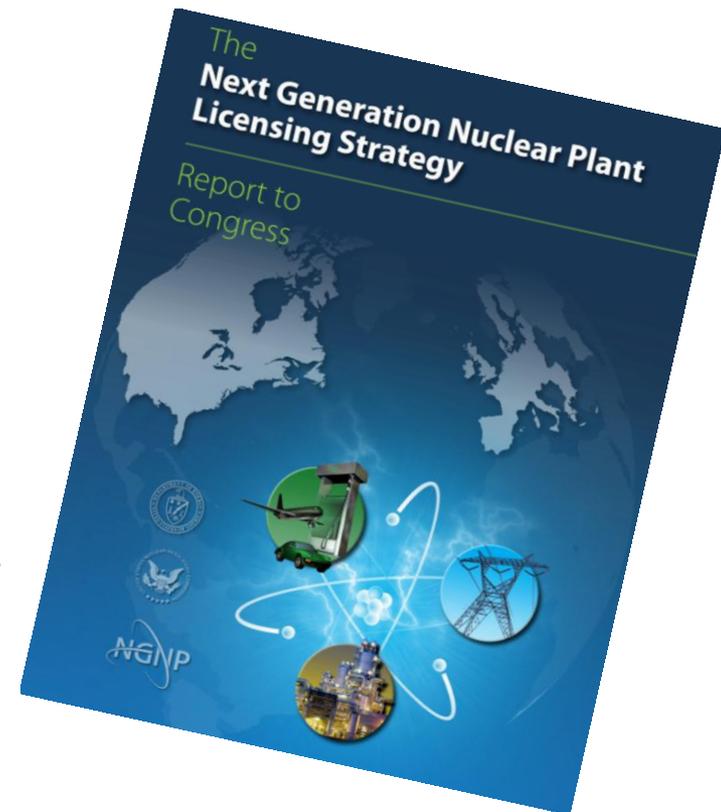
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## NRC-DOE Licensing Strategy – Report to Congress (2008)

NRC and Department of Energy (DOE) jointly agreed, rather than establishing a new set of regulations covering High Temperature Gas-Cooled Reactors (HTGRs), that:

***“The best approach to establish the licensing and safety basis for the NGNP will be to develop a risk-informed and performance-based technical approach that adapts existing NRC LWR technical licensing requirements in establishing NGNP design-specific technical licensing requirements”***



# Two Pronged Approach to Safety Design Criteria

- 1) Adaptation of LWR-based NRC 10CFR50 App A General Design Criteria (GDCs) to modular HTGRs
  - Joint initiative between DOE and NRC
  - Draft design criteria have been developed by DOE & National Laboratories
  - Review by reactor developers and end-users is in progress
  - DOE expects to issue a proposed set of criteria to NRC in November, 2014
  
- 2) Development of Principal Design Criteria (PDC) from a Risk-Informed Approach
  - Establish an approach to demonstrate how the functional statements from top level regulatory criteria are met for:
    - Mitigation of Design Basis Events
    - Prevention of Beyond Design Basis Events

# NRC Feedback on DOE's Proposed Risk-Informed Approach

- DOE has been engaged with the NRC for 4 years (2009 – 2013) in addressing priority NGNP licensing topics
  - White paper submittals from DOE/INL to NRC
  - Written responses to NRC requests for information
  - Numerous NRC public meetings
  
- NRC has completed a draft assessment, and concluded that the following approaches proposed by DOE are reasonable:
  - Establishment of event sequence frequencies
  - Assessment of integrated risk at multi-reactor module plant sites
  - Overall process for performing assessments against the top level regulatory criteria
  - Approach to risk-informed safety classification of structures, systems, and components

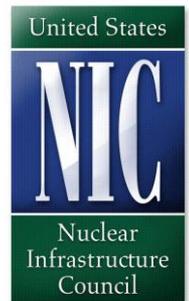


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# NGNP Industry Alliance Limited

- Promoting the development and commercialization of High Temperature Gas-cooled Reactor (HTGR) technology
- HTGR – safe & reliable nuclear energy for industrial applications, reducing dependence on fossil fuels for high temperature heat resources



US ULTRA SAFE  
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TECHNOLOGY  
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PTAC



SGL GROUP  
THE CARBON COMPANY



COMMUNITY REUSE ORGANIZATION

two states, one future

**Advanced Research  
Center**



Wyoming Governor's Office

[Manufacturing Excellence Consulting, Inc.](#)



[www.ngnpalliance.org](http://www.ngnpalliance.org)

## NGNP Alliance Activities

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### ■ Reports prepared for DOE in 2013

- HTGR Market, Risk and Sensitivity Analyses
- Impacts of HTGR Commercialization
- HTGR Commercialization Strategy
- Impacts from other Energy Sectors
- HTGR Siting Issues
- HTGRs for Liquid Fuels and Feedstock
- Industrial Integration
- HTGR Regulatory Risks

### ■ Activities conducted independently by NGNP Alliance

- Studies of Wyoming and Kentucky nuclear assisted CTL
- Canadian oil sands and Louisiana cogeneration siting studies
- Outreach to Kingdom of Saudi Arabia, Singapore, Indonesia, Thailand
- Collaborations: Japan, Korea, NC2I, StarCore



- Going forward, DOE will continue to support:
  - High priority research, including testing at scaled facilities, supporting modular HTGR safety case and licensing approach
  - International collaborations supporting HTGR technology
  - HTGR licensing framework development activities
  
- US supports this important CRP activity
  - Establish comprehensive process for development of safety design criteria
  - Integrate results and insights obtained from ongoing research supporting HTGR safety case
  - Build on US experience with NNGP, including positive initial response from NRC regarding DOE's proposed process for design criteria development