

4. AGENDA

INTERNATIONAL WORKING GROUP ON HIGH TEMPERATURE REACTORS

SPECIALISTS' MEETING ON

"MECHANICAL BEHAVIOUR OF GRAPHITE FOR HTR's"

GIF-SUR-YVETTE, FRANCE, 11-13 JUNE 1979

MONDAY

- 9:00 Opening Remarks; Dr. M.L. Paterin, Chairman
- 10:00 DESIGN CRITERIA, Dr. G. Kleist, Session Chairman
- A-1) A.N. KNOWLES, "Criteria for the selection of graphites for HTR integral block fuel elements".
- A-2) N. PRINCE, "A comparison of predicted and measured graphite moderator behaviour during 16 years operation of the windscale advanced gas-cooled reactor".
- 12:00 Lunch
- 14:30 DESIGN CRITERIA (Cont.), Dr. M. Prince, Session Chairman
- A-3) J.G. WILLASCHEK, "Analysis of nuclear graphite core components in HTRs and a critique on conventional approaches".
- A-4) J.E. BROCKLEHURST, B.T. KELLY, "Graphite structure and its relationship to mechanical engineering design".
- 16:00 STRESS ANALYSIS COMPARISON WITH EXPERIMENTS, Dr. N. Knowles, Session Chairman
- B-1) A. SCHMIDT, "Analysis of the stresses in a side reflector block of pebble bed reactor".
- B-2) G. JOUQUET, Y. BERTHION, A. L'HOMME, "Studies on the graphite rupture under irradiation-induced strains".

THE PROBABILITY OF FAILURE OF EACH SPECIMEN TYPE CAN BE PREDICTED WITHIN THE LIMITS OF ONE STANDARD DEVIATION

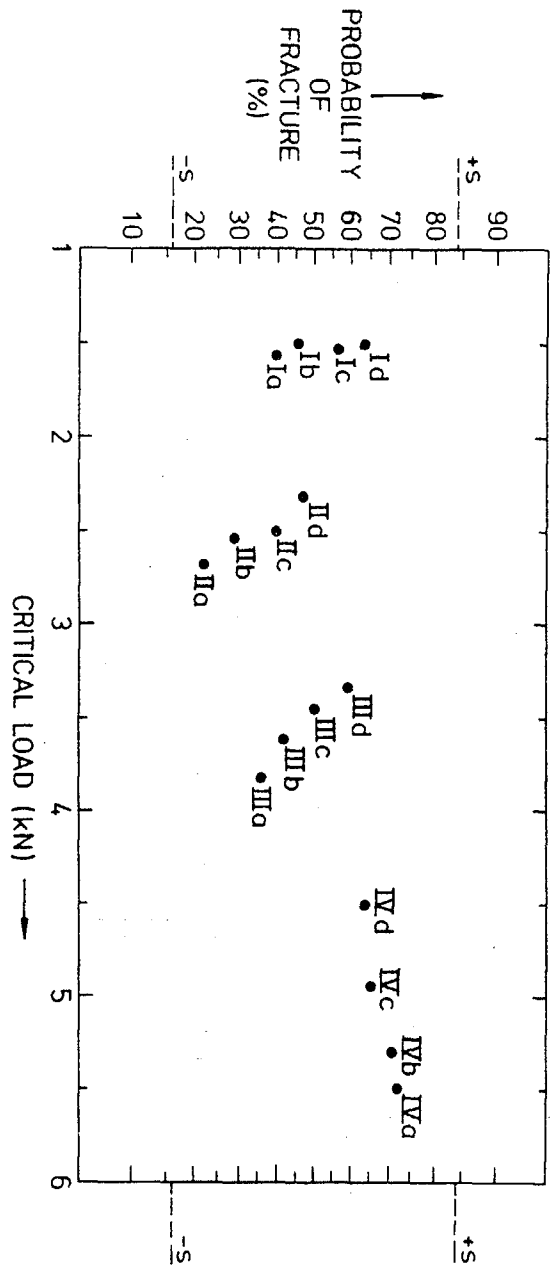


Fig. 4: Mean probabilities of fracture for 16 different specimen types as a function of fracture load (s = standard deviation)

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