

ABBREVIATIONS

ANL	Argonne National Laboratory
ATWS	anticipated transients without scram
BDBA	beyond DBA
BC	begin of cycle
BOEC	begin of equilibrium cycle
BOP	balance of plant
CARR	Chinese advanced research reactor
CDA	core disruptive accident
CDF	core damage factor
CEFR	China experimental fast reactor
CFD	computational fluid dynamics
CHST	clad hot spot temperature
CIAE	China Institute of Atomic Energy
CRBRP	clinch river breeder reactor
CRIEPI	Central Research Institute of Electric Power Industry
CSA	core subassemblies
CSR	control and safety rod
CSRDM	control and safety rod drive mechanism
CSS	core support structure
DBA	design base accident
DBE	design basis event
DEGR	double ended guillotine rupture
DF	driver fuel
DN	equivalent diameter
DSL	design safety limits
DSR	diverse safety rod
DSRDM	diverse safety rod drive mechanism
EFD	European Fast Reactor
EFR	European Fast Reactor
EOEC	end of equilibrium cycle
FAs	fuel assemblies
FBR	fast breeder reactor
FBTR	fast breeder test reactor
FFTF	fast flux test facility
FSA	fuel sub-assembly
GEM	gas expansion module
GIF	Generation IV International Forum

GP	grid plate
HEFS	high energy fluid system
IB	internal blanket
IGCAR	Indira Gandhi Centre for Atomic Research
IHTS	intermediate heat transport system
IHX	intermediate heat exchangers
INPRO	International Project on Innovative Nuclear Reactors and Fuel Cycles
ISI	In Service Inspection
IVSs	in-vessel storages
JAERI	Japan Atomic Energy Research Institute
JIS	Japanese Industrial Standard
JNC	Japan Nuclear Cycle Development Institute
KAEC	Korea Atomic Energy Commission
KAERI	Korea Atomic Energy Research Institute
KALIMER	Korean Advanced Liquid Metal Reactor
LoD	lines of defence
LBB	leak before break
LIPOSO	Llaison POMpe SOMmier
LMFBR	liquid metal cooled fast breeder reactor
LOCA	loss of coolant accident
LOF	loss of flow
MCP	main circulation pump
MEFS	moderate energy fluid system
MV	main vessel
NPSH	net positive suction head
NPSH _A	available net positive suction head
NPSH _R	required net positive suction heas
NRC	Nuclear Regulatory Commission
NSSS	nuclear steam supply system
OKBM	Experimental Designing Bureau of Machine Building
PCP	primary coolant piping
PFBR	prototype fast breeder reactor
PHTS	primary heat transport system
PLC	pipe leak-tightness control
PSAR	preliminary safety analysis report
PSDRS	passive safety decay heat removal system
PSP	primary sodium pumps
RA	reactor assembly
RB	radial blanket

RCB	reactor containment building
RCC-MR	règles de conception et de construction des matériels mécaniques des îlots nucléaires
SA	sub-assembly
SASHST	sub-assembly sodium hot spot temperature
SFR	sodium-cooled fast reactor
SG	steam generators
SGS	steam generator system
SSE	single stress event
T _{Cl}	cladding hotspot temperature
T _{Na}	sodium hotspot temperature
TS	top shield
ULOF	unprotected loss of flow
ULOHS	unprotected loss of heat sink
USS	ultimate shutdown system
UTOP	unprotected transient over power
VHTR	very high temperature reactor